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US Nuclear Regulatory Commission
Washington, DC 20555

Attention: Document Control Desk

Three Mile Island Nuclear Station, Unit 2 (TMI-2)
Operating License No. DPR-73
Docket No. 50-320
Response to Generic Letter 89-21

Dear Sirs:

In response to Generic Letter 89-21, attached is a completed tabulation of Unresolved Safety Issues (USI) for TMI-2. As expected, most of the USIs do not apply to TMI-2 in its current shutdown condition. However, as stated in the attached, if a decision is made to restart the plant, these issues will be reviewed for applicability.

Sincerely,

M B Roche

M. B. Roche
Director, TMI-2

EDS/emf

Attachment

cc: W. T. Russell - Regional Administrator, Region I
J. F. Stolz - Director, Plant Directorate I-4
L. H. Thonus - Project Manager, TMI Site
F. I. Young - Senior Resident Inspector, TMI

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UNRESOLVED SAFETY ISSUES FOR WHICH A FINAL TECHNICAL RESOLUTION HAS BEEN ACHIEVED

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-1	Water Hammer	SECY 84-119 NUREG-0927, Rev. 1 NUREG-0993, Rev. 1 NUREG-0737 Item I.A.2.3 SRP revisions	A11	N/A	This issue will be reviewed for applicability prior to restart.
A-2/ MPA D-10	Asymmetric Blowdown Loads on Reactor Primary Coolant Systems	NUREG-0609 GL 84-04, GDC-4	PWR	N/A	This issue will be reviewed for applicability prior to restart. (Ref. NRC letter dated January 30, 1985)
A-3	Westinghouse Steam Generator Tube Integrity	NUREG-0844 SECY 86-97 SECY 88-272 GL 85-02 (No requirements)	W-PWR	N/A	
A-4	CE Steam Generator Tube Integrity	NUREG-0844, SECY 86-97 SECY 88-272 GL 85-02 (No requirements)	CE-PWR	N/A	
A-5	B&W Steam Generator Tube Integrity	NUREG-0844, SECY 86-97 SECY 88-272 GL 85-02 (No Requirements)	B&W-PWR	N/A	This issue will be reviewed for applicability prior to restart. (Ref. NRC Letter dated July 17, 1985)
E A-6	Mark I Containment Short-Term Program	NUREG-0408	Mark I-BWR	N/A	

* C - COMPLETE
 NC - NO CHANGES NECESSARY
 NA - NOT APPLICABLE
 I - INCOMPLETE
 E - EVALUATING ACTIONS REQUIRED

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-7/ D-01	Mark I Long-Term Program	NUREG-0661 NUREG-0661 Suppl. 1 GL 79-57	Mark I-BWR	N/A	
A-8	Mark II Containment Pool Dynamic Loads	NUREG-0808 NUREG-0487, Suppl. 1/2 NUREG-0802 SRP 6.2.1.1C GDC 16	Mark II-BWR	N/A	
A-9	Anticipated Transients Without Scram	NUREG-0460, Vol. 4 10 CFR 50.62	All	N/A	This issue will be reviewed for applicability prior to restart.
A-10/ MPA B-25	BWR Feedwater Nozzle Cracking	NUREG-0619 Letter from DG Eisenhut dated 11/13/80 GL 81-11	BWR	N/A	
A-11	Reactor Vessel Material Toughness	NUREG-0744, Rev. 1 10 CFR 50.60/ 82-26	All	N/A	This issue will be reviewed for applicability prior to restart. (Ref. NRC Letter dated February 3, 1983)
A-12	Fracture Toughness of Steam Generator and Reactor Coolant Pump Supports	NUREG-0577, Rev. 1 SRP Revision 5.3.4	PWP	N/A	This issue will be reviewed for applicability prior to restart.
A-17	Systems Interactions	Ltr: DeYoung to licensees - 9/77 NUREG-1174, NUREG- 1229, NUREG/CR-3922, NUREG/CR-4261, NUREG/ CR-4470, GL 89-18 (No requirements)	All	N/A	This issue will be reviewed for applicability prior to restart.
A-24/ MPA B-60	Qualification of Class 1E Safety-Related Equipment	NUREG-0588, Rev. 1 SRP 3.11 10 CFR 50.49 GL 82-09, GL 84-24 GL 85-15	All	N/A	TMI-2 is not required to comply with 10 CFR 50.49 until 6 months prior to restart. (Ref. NRC Letter dated July 22, 1983)

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A-26/ MPA B-04	Reactor Vessel Pressure Transient Protection	DOP Letters to Licensees 8/76 NUREG-0224 NUREG-0371 SRP 5.2 GL 88-11	PWR	N/A	This issue will be reviewed for applicability prior to restart.
A-31	Residual Heat Removal Shutdown Requirements	NUREG-0606 RG 1.113, RG 1.139 SRP 5.4.7	All OLs After 01/79.	N/A	This issue will be reviewed for applicability prior to restart.
A-36/ C-10, C-15	Control of Heavy Loads Near Spent Fuel	NUREG-0612 SRP 9.1.5 GL 81-07, GL 83-42, GL 85-11 Letter from DG Eisenhut dated 12/22/80	All	C	A safety evaluation is provided to NRC for each major lifting activity prior to the lift. In addition, this will no longer be a concern upon entry into Facility Mode 2.
A-39	Determination of SRV Pool Dynamic Loads and Pressure Transients	NUREG-0802 NUREGs-0763,0783,0802 NUREG-0661 SPP 6.2.1.1.C	BWR	N/A	
A-40	Seismic Design Criteria	SRP Revisions, NUREG/ CR-4776, NUREG/CR-0054, NUREG/CR-3480, NUREG/ CR-1582, NUREG/CR-1161, NUREG-1233, NUREG-4776 NUREG/CR-3805 NUREG/CR-5347 NUREG/CR-3509	All	N/A	
A-42/ MPA B-05	Pipe Cracks in Boiling Water Reactors	NUREG-0313, Rev. 1 NUREG-0313, Rev. 2 GL 81-03, GL 88-01	BWR	N/A	

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A-43	Containment Emergency Sump Performance	NUREG-0510, NUREG-0869, Rev. 1 NUREG-0897, R.G.1.82 (Rev. 0), SRP 6.2.2 GL 85-22 No Requirements	A11	N/A	This issue will be reviewed for applicability prior to restart. (Ref. NRC Letter dated February 18, 1986)
A-44	Station Blackout	RG 1.155 NUREG-1032 NUREG-1109 10 CFR 50.63	A11	N/A	This issue will be reviewed for applicability prior to restart.
A-45	Shutdown Decay Heat Removal Requirements	SECY 88-260 NUREG-1289 NUREG/CR-5230 SECY 88-260 (No requirements)	A11	N/A	This issue will be reviewed for applicability prior to restart.
A-46	Seismic Qualification of Equipment in Operating Plants	NUREG-1030 NUREG-1211/ GL 87-02, GL 87-03	A11	N/A	This issue will be reviewed for applicability prior to restart. (Ref. NRC Letter dated May 5, 1987)
A-47	Safety Implication of Control Systems	NUREG-1217, NUREG- 1218 GL 89-19	A11	N/A	This issue will be reviewed for applicability prior to restart.
A-48	Hydrogen Control Measures and Effects of Hydrogen Burns on Safety Equipment	10 CFR 50.44 SECY 89-122	A11, except PWRs with large dry containments	N/A	This issue will be reviewed for applicability prior to restart.
A-49	Pressurized Thermal Shock	PGs 1.154, 1.99 SECY 82-465 SECY 83-288 SECY 81-687 10 CFR 50.61/ GL 88-11	PWR	N/A	This issue will be reviewed for applicability prior to restart. (Ref. NRC Letter dated December 30, 1985)